

chapter 31: Dosimetry

How can the activity be measured?

intensity (activity) of the emission $\rightsquigarrow \left| \frac{dN}{dt} \right| = R$
effect a given type of radiation has on human body
 \therefore Much more important is the absorbed radiation dose

The SI unit, gray (Gy) $\rightarrow 1 \text{ Gy} = 1 \text{ J/kg}$

non SI unit, Rad $\rightarrow 1 \text{ Gy} = 100 \text{ RAD}$

energy deposited per unit mass of material

earliest non SI unit
 $1 \text{ R} = 0.01 \text{ Gy} = 0.01 \text{ J/kg}$

Relative Biological Effectiveness (RBE) \rightarrow the number of rads of x-Ray or Grays that produces the same biological damage

effective dose \rightarrow product of the dose (physical absorbed one) times to the RBE factor

effective dose

$$E_d = A D \times RBE$$

absorbed dose

Relative Biological Effectiveness

SI unit for the effective dose \rightarrow Sievert (Sv)

non SI unit \rightarrow rem

1 Sv = 100 rem

31-5 Dosimetry

38. (I) $350 \overset{AD}{\text{rads}}$ of α -particle radiation is equivalent to how many rads of X-rays in terms of biological damage?

$$\begin{aligned} 350 \alpha &\rightarrow \text{rad X-Rays} \\ 1 \alpha &\rightarrow 20 \text{ rad} \end{aligned}$$

7000 Rad of X-Ray

40. (II) How much energy is deposited in the body of a 65-kg adult exposed to a 2.5-Gy dose?

$$AD = \frac{E}{m} \rightarrow 2.5 = \frac{E}{65} \quad E = 162.5 \text{ J}$$

41. (II) A cancer patient is undergoing radiation therapy in which protons with an energy of 1.2 MeV are incident on a 0.20-kg tumor. (a) If the patient receives an effective dose of 1.0 rem, what is the absorbed dose? (b) How many protons are absorbed by the tumor? Assume RBE \approx 1.

$$E \rightarrow 1.2 \times 10^6 \times 1.602 \times 10^{-19} \text{ J}$$

$$ED \rightarrow 1 \text{ rem}$$

$$m \rightarrow 0.2 \text{ kg}$$

$$\text{(a) } ED = AD * RBE$$

$$1 = AD * 1$$

$$AD = 1 \text{ Rad}$$

$$\hookrightarrow 0.01 \text{ Gy}$$

$$\begin{aligned} &0.01 \text{ Gy} * \frac{1 \text{ J/kg}}{1 \text{ Gy}} \\ &* 0.2 \text{ kg} * \frac{1 \text{ eV}}{1.602 \times 10^{-19} \text{ J}} \end{aligned}$$

$$* \frac{P}{1.2 \times 10^6 \text{ eV}}$$

$$P = 1.01 \times 10^{10}$$

46. (II) $^{57}_{27}\text{Co}$ emits 122-keV γ rays. If a 65-kg person swallowed 1.55 μCi of $^{57}_{27}\text{Co}$, what would be the dose rate (Gy/day) averaged over the whole body? Assume that 50% of the γ -ray energy is deposited in the body. [Hint: Determine the rate of energy deposited in the body and use the definition of the gray.]

energy per decay

50%

يعني ببس
50% بيأخذ

$$122 \times 10^3 \times 1.602 \times 10^{-19} \frac{\text{energy}}{\text{decay}} \times 1.55 \times 10^{-6} \times 3.7 \times 10^{10} \frac{\text{decay}}{\text{second}} = 1.12 \times 10^9 \frac{\text{energy}}{\text{sec}}$$

energy per decay activity decay second

Absorbed dose = $\frac{\text{energy}}{\text{mass}}$ ← Absorbed dose ← $\frac{\text{Gy}}{\text{day}}$ ← طلب

← $\frac{1}{\text{day}}$ ← time ← day

$$\frac{1.12 \times 10^9 \text{ energy}}{65 \text{ kg}} \times \frac{1 \text{ sec} \times 1 \text{ day}}{3600 \times 24} \times 0.5 = 7.44 \times 10^{-7} \text{ gray/day}$$

↓ the person only absorbs 50%

47. (II) Ionizing radiation can be used on meat products to reduce the levels of microbial pathogens. Refrigerated meat is limited to 4.5 kGy . If 1.6-MeV electrons irradiate 5 kg of beef, how many electrons would it take to reach the allowable limit?

$$AD = \frac{\text{Energy}}{\text{mass}} \rightarrow \frac{\text{J}}{\text{kg}} \times \text{kg} \times \frac{\text{eV}}{\text{J}} \times \frac{e^-}{1 \text{ eV}}$$

allowable limit

$$4.5 \times 10^3 \frac{\text{J}}{\text{kg}} \times 5 \text{ kg} \times \frac{1 \text{ eV}}{1.602 \times 10^{-19} \text{ J}} \times \frac{\text{num of } e^-}{1.6 \times 10^6 \text{ eV}}$$

$$= 8.77 \times 10^{16} \approx 9 \times 10^{16} e^-$$

44. (II) A 1.6-mCi source of $^{32}_{15}\text{P}$ (in NaHPO_4), a β emitter, is implanted in a tumor where it is to administer 32 Gy . The half-life of $^{32}_{15}\text{P}$ is 14.3 days, and 1.0 mCi delivers about 10 mGy/min . Approximately how long should the source remain implanted?

dose

$$\text{rate} \Rightarrow 1 \text{ mCi} \rightarrow 10 \text{ mGy/min}$$

$$1.6 \text{ mCi} \rightarrow 2 \text{ Gy/min}$$

$$\text{dose} = \frac{\text{time} \times \text{rate}}{??}$$

$$\text{time} = \frac{\text{dose}}{\text{rate}}$$

$$= \frac{32 \text{ Gy}}{10 \times 10^{-3} \text{ Gy/min}}$$

$$\frac{32 \text{ min}}{1.6 \times 10 \times 10^{-3}} \times \frac{1 \text{ hour}}{60 \text{ min}} \times \frac{1 \text{ day}}{24 \text{ hour}}$$

$$= 1.38 \text{ days}$$

R (III) Assume a liter of milk typically has an activity of 2000 pCi due to $^{40}_{19}\text{K}$. If a person drinks two glasses (0.5 L) per day, estimate the total effective dose (in Sv and in rem) received in a year. As a crude model, assume the milk stays in the stomach (12 hr) and is then released. Assume also that roughly 10% of the 1.5 MeV released per decay is absorbed by the body. Compare your result to the normal allowed dose of 100 mrem per year. Make your estimate for (a) a 60-kg adult, and (b) a 6-kg baby.

$$a) \quad ED = AD \times RBE$$

$$\left\{ \begin{array}{l} \text{total E} \\ \text{kg} \end{array} \right\} \times RBE$$

$$1.5 \times 10^6 \times 1.6 \times 10^{-19} \frac{\text{J}}{\text{decay}} \times 2000 \times 10^{-12} \times 3.7 \times 10^{10} \frac{\text{decay}}{\text{s}} \times 12 \text{ hr} \times \frac{3600 \text{ s}}{1 \text{ hr}} \times 0.1$$

$$\times 0.5 \frac{\text{L}}{\text{day}} \times 365 \frac{\text{days}}{\text{year}}$$

$$= 1.4 \times 10^{-5} \frac{\text{J}}{\text{year}}$$

$$1.4 \times 10^{-5} \frac{\text{J}}{\text{year}} \times \frac{1}{60 \text{ kg}} \times 1 = 2.33 \frac{\text{Sv}}{\text{year}} \rightarrow 0.023 \frac{\text{mrem}}{\text{year}}$$

6. Ionizing radiation can be used on meat products to reduce the levels of microbial pathogens. Assume that for refrigerated meat the upper allowed limit is 3.8 kGy . If a beam of electrons, each of energy 1.6 MeV irradiates 3.0 kg of beef, how many electrons should the mass of beef absorb to reach the upper allowed limit?

- A. 4.5×10^{16}
 B. 4.5×10^{10}
 C. 3.8×10^{16}
 D. 3.8×10^{10}
 E. 1.6×10^{10}

$$\frac{J}{kg} \times kg \times \frac{eV}{1J} \times \frac{e^-}{eV}$$

$$3.8 \times 10^3 \frac{J}{kg} \times 3 kg \times \frac{1eV}{1.6 \times 10^{19} J} \times \frac{\text{num of } e^-}{1.6 \times 10^6 eV} = 4.47 \times 10^{16} e^-$$

Answer: A

7. A biological tissue of mass m is exposed to 90 rad of alpha radiation. How many rads of slow neutrons can cause the same biological damage to the same tissue? (For alpha $RBE=20$, for slow neutrons $RBE=5$).

- A. 20
 B. 100
 C. 90
 D. 360
 E. 1800

$$90 \text{ rad} \times 20 \alpha \rightarrow 5 \text{ rad} \times x$$

$$\frac{90 \times 20}{5} = 360$$

Answer: D

4. A 70-kg researcher absorbs 4.5×10^8 neutrons in a workday, each of energy 1.2 MeV . The relative biological effectiveness (RBE) for these neutrons is 10 . What is the equivalent dosage of the radiation exposure for this researcher, in mrem?

- A. 3.7
 B. 0.39
 C. 0.77
 D. 1.2
 E. 12

$$ED = AD \times RBE$$

$$\frac{\text{energy neutron}}{70} \leftarrow \frac{1.2 \times 10^6 \times 1.6 \times 10^{19} \times 4.5 \times 10^8}{70} \times 10 = 1.23 \text{ Sv} \times 10^3 \times 100 = 1.23 \text{ mrem}$$

Answer: D

5. A 3.0-mCi source of ^{32}P is implanted in a tumor to give it a 24-Gy dose. The half-life of ^{32}P is 14.3 days, and 1 mCi delivers 10 mGy/min . How long (in min) should the source remain implanted?

- A. 143
 B. 300
 C. 240
 D. 720
 E. 800

$$1 \text{ mCi} \rightarrow \frac{10 \text{ mGy}}{\text{min}}$$

$$3 \text{ mCi} \rightarrow \frac{30 \text{ mGy}}{\text{min}}$$

$$\frac{30 \text{ mGy}}{\text{min}}$$

$$\text{rate} \times \text{time} = \text{dose}$$

$$\text{time} = \frac{\text{dose}}{\text{Rate}}$$

$$\text{time} = \frac{24 \text{ Gy}}{30 \times 10^{-3} \text{ Gy/min}}$$

$$\frac{24 \times 10^3 \text{ Gy}}{30 \times 10^{-3} \text{ Gy/min}}$$

$$= 800 \text{ min}$$

Answer: E

$$1 \text{ gray} = 100 \text{ rad}$$

Q2) A 55-kg person has absorbed a 20-rad dose. How many joules of energy are deposited in his body?

- A) 1.1
- B) 20
- C) 11**
- D) 55
- E) 1100

$$\frac{E}{\text{kg}} = AD \quad \text{P is gray}$$

$$\frac{E}{55} = \frac{20}{100} \rightarrow 11 \text{ J}$$

6) A 63-kg researcher absorbs 2.6×10^8 neutrons in a work day. The energy of each neutron is 6.5 MeV. The quality factor (QF) for fast neutrons is 10. The biologically equivalent dosage of the radiation, in mrem (mrem = 10^{-3} rem), is closest to (Note: 1 rad = 0.01 J/kg and 1ev = 1.6×10^{-19} J)

- A) 43
- B) 1.3
- C) 2.7
- D) 13
- E) 4.3**

$$ED = AD \times RBE$$

$$\frac{\text{J}}{\text{kg}} \times RBE \rightarrow \frac{6.5 \times 10^6 \times 1.6 \times 10^{-19} \times 2.6 \times 10^8 \times 10}{63}$$

$$= 4.29 \times 10^5 \text{ Sv} \times 100 \times 10^3 = 4.29 \text{ mrem}$$

15) A radioactive source emits 2.4 MeV neutrons at the rate of 9200 neutrons per second. The number of atoms in the source is 4.0×10^9 . The activity of the source, in nCi, is closest to: Hint (nCi = 10^{-9} Ci) and (1Ci = 3.70×10^{10} decays/sec.)

- A) 2500
- B) 92
- C) 920
- D) 25
- E) 250**

activity \rightarrow $\frac{\text{decay}}{\text{sec}}$

$$9200 \frac{\text{decay}}{\text{sec}} \times \frac{1 \text{ sec}}{3.7 \times 10^{10} \text{ decay}} \text{ Ci}$$

$$= 2.48 \times 10^{-7} \text{ Ci}$$

$$248 \times 10^{-9} \text{ Ci}$$

$$248 \text{ nCi}$$

14) The maximum permissible workday dose for occupational exposure to radiation is 26 mrem. A 63 kg laboratory technician absorbs 2.1 mJ of 0.7 MeV gamma rays in a work day. The quality factor (QF) for gamma rays is 1.0. The ratio of the equivalent dosage received by the technician to the maximum permissible equivalent dosage is closest to: (mrem = 10^{-3} rem, 1rad = 0.01 J/kg and 1ev = 1.6×10^{-19} J)

- A) 0.18 B) 0.14 C) 0.17 **D) 0.13** E) 0.15

$$ED = AD \times RBE$$

$$\frac{2.1 \times 10^{-3}}{63} \times 1 = 3.33 \times 10^{-5} \text{ Sv} \times 100 \times 10^3 = 3.33 \text{ mRem}$$

$$\text{Ratio} \rightarrow \frac{\text{absorbed}}{\text{max}} = \frac{3.33}{26} = 0.128$$

17) The radioactive nuclide ^{60}Co is widely used in medical applications. It undergoes beta decay, and the energy of the decay process is 2.82 MeV per decay event. The half-life of this nucleus is 272 days. Suppose that a patient is given a dose of 6.9 microCurie of ^{60}Co . If all of this material decayed while in the patient's body, what would be the total energy (in J) deposited there? Hint: (1Ci = 3.70×10^{10} decays/sec.) and 1ev = 1.6×10^{-19} J.

- A) 3.9** B) 11.0 C) 14.0 D) 8.63×10^{12} E) 4.15×10^6

$$\text{tot Energy} = \text{num of decay} \times \frac{E}{\text{decay}}$$

$$8.68 \times 10^{12} \times 2.82 \times 10^6 \times 1.6 \times 10^{-19}$$

$$= 3.9$$

$$\lambda = \frac{\ln(2)}{T_{1/2}}$$

$$\frac{\ln(2)}{272 \times 24 \times 60 \times 60}$$

$$2.94 \times 10^{-5} \frac{1}{s}$$

$$R = \lambda (N)$$

number of decay

$$R = 6.9 \times 10^3 \times 3.7 \times 10^7 = 2.94 \times 10^8 \frac{\mu}{s}$$

$$N = 8.68 \times 10^{12}$$

سُبْحَانَ اللَّهِ ، وَالْحَمْدُ لِلَّهِ
وَلَا إِلَهَ إِلَّا اللَّهُ ، وَاللَّهُ أَكْبَرُ
وَلَا حَوْلَ وَلَا قُوَّةَ إِلَّا بِاللَّهِ